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ABSTRACT

The analysis of a previous paper [A. Cole and R. Fitzpatrick, Phys. Plasmas **13**, 032503 (2006)] that maps out all of the two-fluid response regimes of a linear tearing layer interacting with an externally generated resonant magnetic perturbation, in a large aspect-ratio tokamak plasma, is generalized to take into account realistic levels of perpendicular particle transport. A new response-regime map is obtained that differs substantially, in many respects, from the old one. The improved analysis is first employed to find all of the two-fluid growth-rate regimes of a non-interacting low-mode-number tearing mode in a large aspect-ratio tokamak plasma. The analysis is then used to determine the scaling of the error-field penetration threshold with machine parameters in large aspect-ratio tokamak plasmas.

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I. INTRODUCTION

Tearing modes are slowly growing instabilities of ideally stable tokamak plasmas that reconnect magnetic field-lines at various resonant surfaces within the plasma, in the process forming magnetic island chains that degrade the plasma confinement.¹ If tearing modes grow to a sufficiently large amplitude, then they can trigger major disruptions.² Tokamak plasmas are observed to be particularly disruption-prone when tearing modes lock (i.e., become stationary in the laboratory frame) to externally generated, resonant magnetic perturbations.³

It is well known that single-fluid, resistive magnetohydrodynamics (MHD) offers a very poor description of tearing mode dynamics in tokamak plasmas. For instance, the strong diamagnetic flows present in such plasmas decouple the electron and ion flows to some extent, necessitating a two-fluid treatment.⁴ Moreover, resistive-MHD does not take the important ion sound radius lengthscale, below which electron and ion dynamics are further decoupled, into account.^{5,6} Previously, Cole and Fitzpatrick⁷ used the four-field model of Fitzpatrick and Waelbroeck⁸ (which is based on the original four-field model of Hazeltine *et al.*⁹) to determine the two-fluid response of a linear tearing layer to an externally generated, resonant magnetic perturbation in a large aspect-ratio tokamak plasma. However, the treatment of Cole and Fitzpatrick fails to take into account the anomalously large perpendicular particle diffusivity present in tokamak plasmas. The aim of this paper is to correct this deficiency.

This paper is organized as follows. After some preliminary analysis in Sec. II, our fundamental four-field resonant plasma response model is introduced in Sec. III. This model is a slightly improved version of the model derived in Ref. 8 that takes the thermal force into account. (The thermal force merely gives rise to an additional term in the perpendicular electron fluid velocity that scales as the electron temperature gradient.) The model is converted into a set of linear layer equations in Sec. IV. These layer equations are then solved in Sec. V in order to map out all of the linear response regimes of a resonant layer to an externally generated, resonant magnetic perturbation in a large aspect-ratio tokamak plasma. In Sec. VI, we reuse the analysis of Sec. V to map out all of the growth-rate regimes for a low-mode-number (but not $m = 1$) tearing mode in a large aspect-ratio tokamak plasma. Finally, in Sec. VII, we employ the results of Sec. V to determine the scaling of the error-field penetration threshold with machine parameters in large aspect-ratio tokamak plasmas.

II. PRELIMINARY ANALYSIS

A. Plasma equilibrium

Consider a large aspect-ratio tokamak plasma equilibrium whose magnetic flux-surfaces map out (almost) concentric circles in the poloidal plane. Such an equilibrium can be approximated as a periodic cylinder.¹⁰ Let r , θ , z be right-handed cylindrical coordinates. The magnetic axis corresponds to $r = 0$, and the plasma boundary to $r = a$.

The system is assumed to be periodic in the z direction with periodicity length $2\pi R_0$, where R_0 is the simulated major radius of the plasma. The safety-factor profile takes the form $q(r) = r B_z / [R_0 B_\theta(r)]$, where B_z is the constant “toroidal” magnetic field-strength, and $B_\theta(r)$ is the poloidal magnetic field-strength. The standard large aspect-ratio orderings, $r/R_0 \ll 1$ and $B_\theta/B_z \ll 1$, are adopted.

B. Asymptotic matching

Consider a tearing mode perturbation that has m periods in the poloidal direction, and n periods in the toroidal direction. As is well known, the response of the plasma to the tearing mode is governed by marginally stable, ideal-MHD everywhere in the plasma, apart from a (radially) narrow layer centered on the so-called resonant surface, minor radius r_s , at which $q(r_s) = m/n$.¹¹

The perturbed magnetic field associated with the tearing mode is written $\delta\mathbf{B} \simeq \nabla\delta\psi \times \mathbf{e}_z$, where $\delta\psi(r, \theta, \varphi, t) = \delta\psi(r, t) \exp[i(m\theta - n\varphi)]$, and $\varphi = z/R_0$ is a simulated toroidal angle.

In the so-called outer region (i.e., everywhere in the plasma apart from the resonant layer), the perturbed helical magnetic flux, $\delta\psi(r, t)$, satisfies the cylindrical tearing mode equation,¹

$$\frac{\partial^2 \delta\psi}{\partial r^2} + \frac{1}{r} \frac{\partial \delta\psi}{\partial r} - \frac{m^2}{r^2} \delta\psi - \frac{J_z' \delta\psi}{r(1/q - n/m)} = 0, \quad (1)$$

where $J_z(r) = R_0 \mu_0 j_z(r)/B_z$, and $j_z(r)$ is the equilibrium toroidal current density. Here, $' \equiv d/dr$. In general, the solution of Eq. (1) that satisfies physical boundary conditions at the magnetic axis and the plasma boundary is such that $\delta\psi$ is continuous across the resonant surface, whereas $\partial\delta\psi/\partial r$ is discontinuous. The discontinuity of $\partial\delta\psi/\partial r$ across the resonant surface is indicative of the presence of a helical current sheet at the surface. The complex quantity $\Psi_s(t) = \delta\psi(r_s, t)$ determines the amplitude and phase of the reconnected helical magnetic flux at the resonant surface, whereas the complex quantity,¹⁰

$$\Delta\Psi_s = \left[r \frac{\partial \delta\psi}{\partial r} \right]_{r_{s-}}^{r_{s+}}, \quad (2)$$

parameterizes the amplitude and phase of the helical current sheet flowing at the surface. The solution of the cylindrical tearing mode equation in the outer region, in the presence of an externally generated, resonant magnetic perturbation of the same helicity as the tearing mode, leads to the relation,^{10,11}

$$\Delta\Psi_s = (\Delta'_s) \Psi_s + (-\Delta'_s) \Psi_v, \quad (3)$$

where Δ'_s is a real dimensionless quantity known as the tearing stability index. Moreover, Ψ_v is the so-called vacuum flux and is defined as the reconnected magnetic flux that would be driven at the resonant surface by the resonant magnetic perturbation where the plasma is intrinsically tearing stable (i.e., $\Delta'_s < 0$) and where there is no current sheet at the resonant surface (i.e., $\Delta\Psi_s = 0$).

The current sheet at the resonant surface can only be resolved by solving a two-fluid, resistive-MHD, plasma response model in the so-called inner region (i.e., the region of the plasma in the immediate vicinity of the resonant surface), and asymptotically matching the solution so obtained to the ideal-MHD solution in the outer region. The particular plasma response model used in this paper is described in Sec. III.

III. FOUR-FIELD RESONANT PLASMA RESPONSE MODEL

A. Useful definitions

The plasma is assumed to consist of two species. First, electrons of mass m_e , electrical charge $-e$, number density n , and temperature T_e . Second, ions of mass m_i , electrical charge $+e$, number density n , and temperature T_i . Let $p = n(T_e + T_i)$ be the total plasma pressure.

It is helpful to define $n_0 = n(r_s)$, $p_0 = p(r_s)$

$$\eta_e = \frac{d \ln T_e}{d \ln n} \Big|_{r=r_s}, \quad (4)$$

$$\eta_i = \frac{d \ln T_i}{d \ln n} \Big|_{r=r_s}, \quad (5)$$

$$\tau = \left(\frac{T_e}{T_i} \right)_{r=r_s} \left(\frac{1 + \eta_e}{1 + \eta_i} \right), \quad (6)$$

where $n(r)$, $p(r)$, $T_e(r)$, and $T_i(r)$ refer to the density, pressure, and temperature profiles that are unperturbed by the tearing mode.

For the sake of simplicity, the perturbed electron and ion temperature profiles are assumed to be functions of the perturbed electron number density profile in the immediate vicinity of the resonant surface. In other words, $T_e = T_e(n)$ and $T_i = T_i(n)$. This implies that $p = p(n)$. The MHD velocity, which is the velocity of a fictional MHD fluid, is defined $\mathbf{V} = \mathbf{V}_E + V_{\parallel i} \mathbf{b}$, where \mathbf{V}_E is the $\mathbf{E} \times \mathbf{B}$ drift velocity, $V_{\parallel i}$ is the parallel component of the ion fluid velocity, $\mathbf{b} = \mathbf{B}/|\mathbf{B}|$, and \mathbf{B} is the magnetic field-strength.

B. Fundamental fields

The four fundamental fields in our four-field model—namely, ψ , ϕ , N , and V —have the following definitions:

$$\nabla\psi = \frac{\mathbf{n} \times \mathbf{B}}{r_s B_z}, \quad (7)$$

$$\nabla\phi = \frac{\mathbf{n} \times \mathbf{V}}{r_s V_A}, \quad (8)$$

$$N = -\hat{d}_i \left(\frac{p - p_0}{B_z^2 / \mu_0} \right), \quad (9)$$

$$V = \hat{d}_i \left(\frac{\mathbf{n} \cdot \mathbf{V}}{V_A} \right). \quad (10)$$

Here, $\mathbf{n} = (0, \epsilon/q_s, 1)$, $\epsilon = r/R_0$, $q_s = m/n$, $V_A = B_z / \sqrt{\mu_0 n_0 m_i}$, $d_i = \sqrt{m_i / (n_0 e^2 \mu_0)}$, and $\hat{d}_i = d_i / r_s$. Our model also employs the auxiliary field,

$$J = -\frac{2\epsilon_s}{q_s} + \hat{\nabla}^2 \psi, \quad (11)$$

where $\epsilon_s = r_s/R_0$, and $\hat{\nabla} = r_s \nabla$.

C. Fundamental equations

The four-field model takes the form,⁸

$$\frac{\partial \psi}{\partial t} = [\phi, \psi] - \tau_e (1 + \lambda_e) [N, \psi] + \hat{\eta}_{\parallel} J + \hat{E}_{\parallel}, \quad (12)$$

$$\frac{\partial \hat{\nabla}^2 \phi}{\partial t} = \left[\phi, \hat{\nabla}^2 \phi \right] - \frac{\tau_i}{2} \left(\hat{\nabla}^2 [\phi, N] + [\hat{\nabla}^2 \phi, N] + [\hat{\nabla}^2 N, \phi] \right) + [J, \psi] + \hat{\chi}_\phi \hat{\nabla}^4 (\phi + \tau_i N), \quad (13)$$

$$\frac{\partial N}{\partial t} = [\phi, N] + c_\beta^2 [V, \psi] + \hat{d}_\beta^2 [J, \psi] + \hat{D}_\perp \hat{\nabla}_\perp^2 N, \quad (14)$$

$$\frac{\partial V}{\partial t} = [\phi, V] + [N, \psi] + \hat{\chi}_\phi \hat{\nabla}^2 V. \quad (15)$$

Here, $[A, B] \equiv \hat{\nabla} A \times \hat{\nabla} B \cdot \mathbf{n}$, $\tau_e = [\tau/(1+\tau)]$, $\tau_i = 1/(1+\tau)$, $\hat{t} = t/(r_s/V_A)$, $\hat{\eta}_{\parallel,\perp} = \eta_{\parallel,\perp}^+ / (\mu_0 r_s V_A)$, $\hat{E}_{\parallel} = E_{\parallel}/(B_z V_A)$, $\hat{\chi}_\phi = \chi_\phi / (r_s V_A)$, where $\eta_{\parallel,\perp}$ is the parallel/perpendicular plasma electrical resistivity at the resonant surface, E_{\parallel} the parallel inductive electric field that maintains the equilibrium toroidal plasma current in the vicinity of the resonant surface, and χ_ϕ the anomalous perpendicular ion momentum diffusivity at the resonant surface. Moreover, $d_\beta = c_\beta d_i$, and $\hat{d}_\beta = d_\beta/r_s$, where $c_\beta = \sqrt{\beta/(1+\beta)}$, and $\beta = (5/3) \mu_0 p_0/B_z^2$. Here, d_β is usually referred to as the ion sound radius. Furthermore, $\lambda_e = 0.71 \eta_e/(1+\eta_e)$ originates from the thermal force. Finally,⁸

$$\hat{D}_\perp \equiv \frac{D_\perp}{r_s V_A} = c_\beta^2 \hat{\eta}_\perp + \frac{2}{3} \left(1 - c_\beta^2 \right) \left(\frac{\eta_e \tau_e}{1 + \eta_e} \hat{\chi}_{\perp e} + \frac{\eta_i \tau_i}{1 + \eta_i} \hat{\chi}_{\perp i} \right), \quad (16)$$

where $\hat{\chi}_{\perp e,i} = \chi_{\perp e,i}/(r_s V_A)$, and $\chi_{\perp e,i}$ is the (anomalous) perpendicular electron/ion energy diffusivity in the vicinity of the resonant surface. Note that Cole and Fitzpatrick (2006) neglects the contribution of $\chi_{\perp e,i}$ to the perpendicular particle diffusivity, D_\perp , leading to an anomalously small estimate for D_\perp . It is this deficiency that we aim to correct in the present paper. To be more exact, given that $c_\beta \ll 1$ in conventional tokamak plasmas, and that $\hat{\eta}_\perp \ll \hat{\chi}_{\perp e,i}$ (i.e., magnetic flux typically diffuses across magnetic flux surfaces in tokamak plasmas much more slowly than energy), the neglect of the terms involving $\hat{\chi}_{\perp e,i}$ in the previous equation leads to the erroneous conclusion that $\hat{D}_\perp \ll \hat{\chi}_{\perp e,i}$ (i.e., particles diffuse across magnetic flux-surfaces in tokamak plasmas much more slowly than energy). The inclusion of the terms leads to the correct conclusion that $\hat{D}_\perp \sim \hat{\chi}_{\perp e,i}$ (i.e., particles diffuse across magnetic flux-surfaces at a similar rate to energy).

D. Matching to plasma equilibrium

The unperturbed plasma equilibrium is such that $\mathbf{B} = (0, B_\theta(r), B_z)$, $p = p(r)$, $\mathbf{V} = (0, V_E(r), V_z(r))$, where $V_E(r) \simeq E_r/B_z$ is the (dominant θ -component of the) $\mathbf{E} \times \mathbf{B}$ velocity. Now, the resonant layer is assumed to have a radial thickness that is much smaller than r_s . Hence, we only need to evaluate plasma equilibrium quantities in the immediate vicinity of the resonant surface. Equations (7)–(10) suggest that

$$\psi = \frac{\hat{x}^2}{2\hat{L}_s}, \quad (17)$$

$$\phi = -\hat{V}_E \hat{x}, \quad (18)$$

$$N = -\hat{V}_* \hat{x}, \quad (19)$$

$$V = \hat{V}_\parallel, \quad (20)$$

where $\hat{x} = (r - r_s)/r_s$, $\hat{L}_s = L_s/r_s$, $L_s = R_0 q_s/s_s$, $\hat{V}_E = V_E(r_s)/V_A$, $\hat{V}_* = V_*(r_s)/V_A$, $V_*(r) = (dp/dr)/(en_0 B_z)$ is the (dominant θ -

component of the) diamagnetic velocity, and $\hat{V}_\parallel = \hat{d}_i V_z(r_s)/V_A$. Here, $s_s = s(r_s)$ and $s(r) = d \ln q/d \ln r$. We also have

$$J = -\left(\frac{2}{s_s} - 1\right) \frac{1}{\hat{L}_s}, \quad (21)$$

and $\hat{E}_{\parallel} = (2/s_s - 1)(\hat{\eta}_{\parallel}/\hat{L}_s)$.

IV. LINEAR RESONANT PLASMA RESPONSE MODEL

A. Introduction

The aim of this section is to obtain a set of linear layer equations from the four-field model introduced in Sec. III.

B. Derivation of linear layer equations

In accordance with Eqs. (17)–(21), let us write

$$\psi(\hat{x}, \zeta, \hat{t}) = \frac{\hat{x}^2}{2\hat{L}_s} + \tilde{\psi}(\hat{x}) e^{i(\zeta - \hat{\omega} \hat{t})}, \quad (22)$$

$$\phi(\hat{x}, \zeta, \hat{t}) = -\hat{V}_E \hat{x} + \tilde{\phi}(\hat{x}) e^{i(\zeta - \hat{\omega} \hat{t})}, \quad (23)$$

$$N(\hat{x}, \zeta, \hat{t}) = -\hat{V}_* \hat{x} + \tau_e \tilde{N}(\hat{x}) e^{i(\zeta - \hat{\omega} \hat{t})}, \quad (24)$$

$$V(\hat{x}, \zeta, \hat{t}) = \hat{V}_\parallel + \tau_e \tilde{V}(\hat{x}) e^{i(\zeta - \hat{\omega} \hat{t})}, \quad (25)$$

$$J(\hat{x}, \zeta, \hat{t}) = -\left(\frac{2}{s_s} - 1\right) \frac{1}{\hat{L}_s} + \hat{\nabla}^2 \tilde{\psi}(\hat{x}) e^{i(\zeta - \hat{\omega} \hat{t})}, \quad (26)$$

where $\zeta = m\theta - n\varphi$, $\hat{\omega} = r_s \omega/V_A$, and ω is the frequency of the tearing mode in the laboratory frame. Substituting Eqs. (22)–(26) into Eqs. (11)–(15), and only retaining terms that are first order in perturbed quantities, we obtain the following set of linear equations:

$$-i[\omega - \omega_E - (1 + \lambda_e) \omega_{*e}] \tau_H \tilde{\psi} = -i\hat{x} \left[\tilde{\phi} - (1 + \lambda_e) \tilde{N} \right] + S^{-1} \hat{\nabla}^2 \tilde{\psi}, \quad (27)$$

$$-i(\omega - \omega_E - \omega_{*i}) \tau_H \hat{\nabla}^2 \tilde{\phi} = -i\hat{x} \hat{\nabla}^2 \tilde{\psi} + S^{-1} P_\phi \hat{\nabla}^4 \left(\tilde{\phi} + \frac{\tilde{N}}{\tau} \right), \quad (28)$$

$$-i(\omega - \omega_E) \tau_H \tilde{N} = -i\omega_{*e} \tau_H \tilde{\phi} - i c_\beta^2 \hat{x} \tilde{V} - i \tau_e \hat{d}_\beta^2 \hat{x} \hat{\nabla}^2 \tilde{\psi} = +S^{-1} P_\perp \hat{\nabla}_\perp^2 \tilde{N}, \quad (29)$$

$$-i(\omega - \omega_E) \tau_H \tilde{V} = i\omega_{*e} \tau_H \tilde{\psi} - i\hat{x} \tilde{N} + S^{-1} P_\phi \hat{\nabla}^2 \tilde{V}. \quad (30)$$

Here, $\tau_H = L_s/(m V_A)$ is the hydromagnetic time, $\omega_E = (m/r_s) \times V_E(r_s)$ the $\mathbf{E} \times \mathbf{B}$ frequency, $\omega_{*e} = -\tau_e (m/r_s) V_*(r_s)$ the electron diamagnetic frequency, $\omega_{*i} = \tau_i (m/r_s) V_*(r_s)$ the ion diamagnetic frequency, $S = \tau_R/\tau_H$ the Lundquist number, $\tau_R = \mu_0 r_s^2/\eta_{\parallel}$ the resistive diffusion time, $\tau_\phi = r_s^2/\chi_\phi$ the toroidal momentum confinement time, and $\tau_\perp = r_s^2/D_\perp$. Furthermore, $P_\phi = \tau_R/\tau_\phi$ and $P_\perp = \tau_R/\tau_\perp$ are magnetic Prandtl numbers.

Let us define the stretched radial variable $X = S^{1/3} \hat{x}$. Assuming that $X \sim \mathcal{O}(1)$ in the layer (i.e., assuming that the layer thickness is roughly of order $S^{-1/3} r_s$), and making use of the fact that $S \gg 1$ in conventional tokamak plasmas, Eqs. (27)–(30) reduce to the following set of linear layer equations:⁷

$$-i[Q - Q_E - (1 + \lambda_e) Q_e] \tilde{\psi} = -iX \left[\tilde{\phi} - (1 + \lambda_e) \tilde{N} \right] + \frac{d^2 \tilde{\psi}}{dX^2}, \quad (31)$$

$$-i(Q - Q_E - Q_i) \frac{d^2 \tilde{\phi}}{dX^2} = -iX \frac{d^2 \tilde{\psi}}{dX^2} + P_\phi \frac{d^4}{dX^4} \left(\tilde{\phi} + \frac{\tilde{N}}{\tau} \right), \quad (32)$$

$$-i(Q - Q_E) \tilde{N} = -iQ_e \tilde{\phi} - i c_\beta^2 X \tilde{V} - iD^2 X \frac{d^2 \tilde{\psi}}{dX^2} + P_\perp \frac{d^2 \tilde{N}}{dX^2}, \quad (33)$$

$$-i(Q - Q_E) \tilde{V} = iQ_e \tilde{\psi} - iX \tilde{N} + P_\phi \frac{d^2 \tilde{V}}{dX^2}. \quad (34)$$

Here, $Q = S^{1/3} \omega \tau_H$, $Q_E = S^{1/3} \omega_E \tau_H$, $Q_{e,i} = S^{1/3} \omega_{*e,i} \tau_H$, and $D = S^{1/3} \tau_e^{1/2} \hat{d}_\beta$.

C. Asymptotic matching

The linear layer equations, (31)–(34), possess tearing parity solutions characterized by the symmetry $\tilde{\psi}(-X) = \tilde{\psi}(X)$, $\tilde{\phi}(-X) = -\tilde{\phi}(X)$, $\tilde{N}(-X) = -\tilde{N}(X)$, $\tilde{V}(-X) = \tilde{V}(X)$. If we assume that the asymptotic behavior of the tearing parity layer solutions is such that

$$\tilde{\psi} \rightarrow \psi_0 \left[1 + \frac{\hat{\Delta}}{2} |X| + \mathcal{O}(X^2) \right], \quad (35)$$

as $|X| \rightarrow \infty$, where ψ_0 is an arbitrary constant, then asymptotic matching to the outer solution yields

$$\Delta \Psi_s = (S^{1/3} \hat{\Delta}) \Psi_s. \quad (36)$$

V. LINEAR RESONANT PLASMA RESPONSE REGIMES

A. Introduction

The aim of this section is to solve the linear layer equations derived in Sec. IV to determine the response of a resonant layer to an external perturbation.

B. Fourier transformation

Equations (31)–(34) are most conveniently solved in Fourier transform space.⁷ Let

$$\bar{\phi}(p) = \int_{-\infty}^{\infty} \tilde{\phi}(X) e^{-ipX} dX, \quad (37)$$

etc. The Fourier transformed linear layer equations become

$$-i[Q - Q_E - (1 + \lambda_e) Q_e] \bar{\psi} = \frac{d}{dp} [\bar{\phi} - (1 + \lambda_e) \bar{N}] - p^2 \bar{\psi}, \quad (38)$$

$$-i(Q - Q_E - Q_i) p^2 \bar{\phi} = \frac{d(p^2 \bar{\psi})}{dp} - P_\phi p^4 \left(\bar{\phi} + \frac{\bar{N}}{\tau} \right), \quad (39)$$

$$-i(Q - Q_E) \bar{N} = -iQ_e \bar{\phi} + c_\beta^2 \frac{d\bar{V}}{dp} - D^2 \frac{d(p^2 \bar{\psi})}{dp} - P_\perp p^2 \bar{N}, \quad (40)$$

$$-i(Q - Q_E) \bar{V} = iQ_e \bar{\psi} + \frac{d\bar{N}}{dp} - P_\phi p^2 \bar{V}, \quad (41)$$

where, for a tearing parity solution,

$$\bar{\phi}(p) \rightarrow \bar{\phi}_0 \left[\frac{\hat{\Delta}}{\pi p} + 1 + \mathcal{O}(p) \right], \quad (42)$$

as $p \rightarrow 0$.

Let us ignore the term $c_\beta^2 d\bar{V}/dp$ in Eq. (40). This approximation can be justified *a posteriori*. It is equivalent to the neglect of the contribution of ion parallel dynamics to the linear plasma response in the resonant layer and effectively decouples Eq. (41) from Eqs. (38)–(40).^{7,12} Equations (38)–(40) reduce to⁷

$$\frac{d}{dp} \left[G(p) \frac{dY_e}{dp} \right] - \frac{A(p)}{B(p)} p^2 Y_e = 0, \quad (43)$$

where

$$G(p) = \frac{p^2}{-i[Q - Q_E - (1 + \lambda_e) Q_e] + p^2}, \quad (44)$$

$$A(p) = -(Q - Q_E)(Q - Q_E - Q_i) - i(Q - Q_E - Q_i)(P_\phi + P_\perp)p^2 + P_\phi P_\perp p^4, \quad (45)$$

$$B(p) = -i[Q - Q_E - (1 + \lambda_e) Q_e] + \{P_\perp - i(1 + \lambda_e)(Q - Q_E - Q_i)D^2\}p^2 + [(1 + \lambda_e) + 1/\tau]P_\phi D^2 p^4, \quad (46)$$

and $Y_e = \bar{\phi} - (1 + \lambda_e) \bar{N}$. The boundary conditions are that $Y_e(p)$ is bounded as $p \rightarrow \infty$, and

$$Y_e(p) \rightarrow Y_0 \left[\frac{\hat{\Delta}}{\pi p} + 1 + \mathcal{O}(p) \right], \quad (47)$$

as $p \rightarrow 0$. In the following, we shall assume that $|Q| \sim |Q_E| \sim |Q_e| \sim |Q_i|$, $P_\phi \sim P_\perp \sim P$, $\tau \sim \mathcal{O}(1)$, and $\lambda_e \sim \mathcal{O}(1)$, for the sake of simplicity.

C. Constant- ψ approximation

Let us suppose that there are two layers in p space. In the small- p layer, suppose that Eq. (43) reduces to

$$\frac{d}{dp} \left[\frac{p^2}{-i[Q - Q_E - (1 + \lambda_e) Q_e] + p^2} \frac{dY_e}{dp} \right] \simeq 0, \quad (48)$$

when $p \sim Q^{1/2}$. Integrating directly, we find that

$$Y_e(p) \simeq Y_0 \left[\frac{\hat{\Delta}}{\pi} \left(\frac{1}{p} + \frac{p}{i[Q - Q_E - (1 + \lambda_e) Q_e]} \right) + 1 + \mathcal{O}(p^2) \right], \quad (49)$$

for $p \leq \mathcal{O}(Q^{1/2})$, where use has been made of Eq. (47). The two-layer approximation is equivalent to the well-known constant- ψ approximation.¹¹

In the large- p layer, for $p \gg \mathcal{O}(Q^{1/2})$, we obtain

$$\frac{d^2 Y_e}{dp^2} - \frac{A(p)}{B(p)} p^2 Y_e \simeq 0, \quad (50)$$

with Y_e bounded as $p \rightarrow \infty$. Asymptotic matching to the small- p layer yields the boundary condition,

$$Y_e(p) \simeq Y_0 \left\{ 1 + \frac{\hat{\Delta}}{\pi} \frac{p}{i[Q - Q_E - (1 + \lambda_e) Q_e]} + \mathcal{O}(p^2) \right\}, \quad (51)$$

as $p \rightarrow 0$.

In the various constant- ψ linear response regimes considered in Sec. VD, Eq. (50) reduces to an equation of the form

$$\frac{d^2 Y_e}{dp^2} - C p^m Y_e \simeq 0, \quad (52)$$

where m is real and non-negative, and C is a complex constant. Let $Y_e = \sqrt{p} Z$ and $q = \sqrt{C} p^n / n$, where $n = (m + 2)/2$. The previous equation transforms into a modified Bessel equation of general order,

$$q^2 \frac{d^2 Z}{dq^2} + q \frac{dZ}{dq} - (q^2 + \nu^2) Z = 0, \quad (53)$$

where $\nu = 1/(m + 2)$. The solution that is bounded as $q \rightarrow \infty$ has the small- q expansion

$$K_\nu(q) = \frac{1}{\Gamma(1 - \nu)} \left(\frac{2}{q}\right)^\nu - \frac{1}{\Gamma(1 + \nu)} \left(\frac{q}{2}\right)^\nu + \mathcal{O}(q^{2-\nu}), \quad (54)$$

where $\Gamma(z)$ is a gamma function. A comparison of this expression with Eq. (51) reveals that

$$\hat{\Delta} = \pi (-i [Q - Q_E - (1 + \lambda_e) Q_e]) \frac{\Gamma(1 - \nu)}{\Gamma(\nu)} \nu^{2\nu-1} C^\nu. \quad (55)$$

Note, finally, that $p_* \sim |C|^{-\nu}$, where p_* denotes the width of the large- p layer in p space. This width must be larger than $Q^{1/2}$ (i.e., the width of the small- p layer) in order for the constant- ψ approximation to hold.

D. Constant- ψ linear response regimes

Suppose that $Q \gg P p^2$ and $D^2 p^2 \ll 1$. It follows that $\nu = 1/4$ and

$$C = \frac{[-i(Q - Q_E)] [-i(Q - Q_E - Q_i)]}{-i [Q - Q_E - (1 + \lambda_e) Q_e]}. \quad (56)$$

Hence, we deduce that

$$\hat{\Delta} = \frac{2\pi\Gamma(3/4)}{\Gamma(1/4)} (-i [Q - Q_E - (1 + \lambda_e) Q_e])^{3/4} \times [-i(Q - Q_E)]^{1/4} [-i(Q - Q_E - Q_i)]^{1/4}. \quad (57)$$

This response regime is known as the resistive-inertial (RI) regime, because the layer response is dominated by plasma resistivity and ion inertia.^{4,11,13} The characteristic layer width is $p_* \sim Q^{-1/4}$, which implies that the regime is valid when $P \ll Q^{3/2}$, $Q \gg D^4$, and $Q \ll 1$.

Suppose that $Q \ll P p^2$ and $D^2 p^2 \ll 1$. It follows that $\nu = 1/6$ and $C = P_\phi$. Hence, we deduce that

$$\hat{\Delta} = \frac{6^{2/3} \pi \Gamma(5/6)}{\Gamma(1/6)} (-i [Q - Q_E - (1 + \lambda_e) Q_e]) P_\phi^{1/6}. \quad (58)$$

This response regime is known as the viscous-resistive (VR) regime, because the layer response is dominated by ion perpendicular viscosity and plasma resistivity.^{10,14} The characteristic layer width is $p_* \sim P^{-1/6}$, which implies that the regime is valid when $P \gg Q^{3/2}$, $P \gg D^6$, and $P \ll Q^{-3}$.

Suppose that $Q \gg P p^2$ and $D^2 p^2 \gg 1$. It follows that $\nu = 1/2$ and

$$C = \frac{-i(Q - Q_E)}{(1 + \lambda_e) D^2}. \quad (59)$$

Hence, we deduce that

$$\hat{\Delta} = \frac{\pi (-i [Q - Q_E - (1 + \lambda_e) Q_e]) [-i(Q - Q_E)]^{1/2}}{\sqrt{1 + \lambda_e} D}. \quad (60)$$

This response regime is known as the semi-collisional (SC) regime.^{5,6} The characteristic layer width is $p_* \sim D Q^{-1/2}$, which implies that the regime is valid when $Q \ll D^4$, $P \ll Q^2/D^2$, and $Q \ll D$.

Suppose, finally, that $Q \ll P p^2$ and $D^2 p^2 \gg 1$. It follows that $\nu = 1/4$ and

$$C = \frac{P_\perp}{[(1 + \lambda_e) + 1/\tau] D^2}. \quad (61)$$

Hence, we deduce that

$$\hat{\Delta} = \frac{2\pi\Gamma(3/4)}{\Gamma(1/4)} \frac{(-i [Q - Q_E - (1 + \lambda_e) Q_e]) P_\perp^{1/4}}{[(1 + \lambda_e) + 1/\tau]^{1/4} D^{1/2}}. \quad (62)$$

This response regime is known as the diffusive-resistive regime, because the layer response is dominated by perpendicular particle diffusion and plasma resistivity. The characteristic layer width is $p_* \sim D^{1/2} P^{-1/4}$, which implies that the regime is valid when $P \gg Q^2/D^2$, $P \ll D^6$, and $P \ll D^2/Q^2$.

E. Nonconstant- ψ linear response regimes

Suppose that $p \ll Q^{1/2}$. In this limit, Eq. (43) reduces to

$$\frac{d}{dp} \left(p^2 \frac{dY_e}{dp} \right) - (-i [Q - Q_E - (1 + \lambda_e) Q_e]) \frac{A(p)}{B(p)} p^2 Y_e = 0. \quad (63)$$

In the various nonconstant- ψ regimes considered in this section, the previous equation takes the form

$$\frac{d}{dp} \left(p^2 \frac{dY_e}{dp} \right) - C p^{m+2} Y_e = 0, \quad (64)$$

where m is real and non-negative, and C is a complex constant. Let $U = p Y_e$. The previous equation yields $d^2 U / dp^2 - C p^m U = 0$. This equation is identical in form to Eq. (52), which we have already solved. Indeed, the solution that is bounded as $p \rightarrow \infty$ has the small- p expansion (54), where $q = \sqrt{C} p^n / n$, $n = (m + 2)/2$, and $\nu = 1/(m + 2)$. Matching to Eq. (47) yields

$$\hat{\Delta} = -\pi \frac{\Gamma(\nu)}{\Gamma(1 - \nu)} \nu^{1-2\nu} C^{-\nu}. \quad (65)$$

The layer width in p -space again scales as $p_* \sim |C|^{-\nu}$. This width must be less than $Q^{1/2}$.

Suppose that $Q \gg P p^2$ and $D^2 p^2 \ll 1$. It follows that $\nu = 1/2$ and

$$C = [-i(Q - Q_E)] [-i(Q - Q_E - Q_i)]. \quad (66)$$

Hence, we deduce that

$$\hat{\Delta} = -\frac{\pi}{[-i(Q - Q_E)]^{1/2} [-i(Q - Q_E - Q_i)]^{1/2}}. \quad (67)$$

This response regime is known as the inertial regime, because the layer response is dominated by ion inertia.^{4,10} Note that the plasma response in the inertial regime is equivalent to that of two closely spaced Alfvén resonances that straddle the resonant surface,¹⁵ In fact, it is easily demonstrated that in real space,

$$\hat{\Delta} = \int_{-\infty}^{\infty} \frac{dX}{(Q - Q_E)(Q - Q_E - Q_i) - X^2}, \quad (68)$$

which suggests that the resonances lie at $X = \pm \sqrt{(Q - Q_E)(Q - Q_E - Q_i)}$. The characteristic layer width is $p_* \sim Q^{-1}$, which implies that the regime is valid when $P \ll Q^3$, $Q \gg D$, and $Q \gg 1$.

Suppose that $Q \ll Pp^2$ and $D^2 p^2 \ll 1$. It follows that $\nu = 1/4$ and

$$C = (-i[Q - Q_E - (1 + \lambda_e)Q_e])P_\phi. \quad (69)$$

Hence, we deduce that

$$\hat{\Delta} = -\frac{\pi}{2} \frac{\Gamma(1/4)}{\Gamma(3/4)} (-i[Q - Q_E - (1 + \lambda_e)Q_e])^{-1/4} P_\phi^{-1/4}. \quad (70)$$

This response regime is known as the viscous-inertial regime, because the layer response is dominated by ion perpendicular viscosity and ion inertia.¹³ The characteristic layer width is $p_* \sim Q^{-1/4} P^{-1/4}$, which implies that the regime is valid when $P \gg Q^3$, $P \gg D^4/Q$, and $P \gg Q^{-3}$.

Suppose, finally, that $Q \ll Pp^2$ and $D^2 p^2 \gg 1$. It follows that $\nu = 1/2$ and

$$C = \frac{(-i[Q - Q_E - (1 + \lambda_e)Q_e])P_\perp}{[(1 + \lambda_e) + 1/\tau]D^2}. \quad (71)$$

Hence, we deduce that

$$\hat{\Delta} = -\frac{\pi[(1 + \lambda_e) + 1/\tau]^{1/2} D}{(-i[Q - Q_E - (1 + \lambda_e)Q_e])^{1/2} P_\perp^{1/2}}. \quad (72)$$

This response regime is known as the diffusive-inertial regime, because the layer response is dominated by perpendicular particle diffusivity and ion inertia. The characteristic layer width is $p_* \sim DQ^{-1/2} P^{-1/2}$, which implies that the regime is valid when $Q \ll D$, $P \ll D^4/Q$, and $P \gg D^2/Q^2$.

F. Discussion

Figures 1 and 2 show the extents of the various different linear resonant response regimes in Q - P space for the cases $D < 1$ and $D > 1$, respectively. Note that

$$Q \simeq 1.02 \times 10^2 \times \left[\frac{r_s^{2/3}(\text{m}) L_s^{2/3}(\text{m}) T_0^{1/2}(\text{keV}) n_0^{1/2}(10^{19} \text{m}^{-3}) \omega(\text{krad/s})}{m^{2/3} B_z^{2/3}(\text{T})} \right], \quad (73)$$

$$P \simeq 5.04 \times 10^1 \left[\chi_\perp(\text{m}^2/\text{s}) T_0^{3/2}(\text{keV}) \right], \quad (74)$$

$$D \simeq 4.15 \left[\frac{\tau_e^{1/2} m^{1/3} T_0(\text{keV})}{r_s^{1/2}(\text{m}) L_s^{1/2}(\text{m}) B_z^{1/6}(\text{T}) n_0^{1/6}(10^{19} \text{m}^{-3})} \right], \quad (75)$$

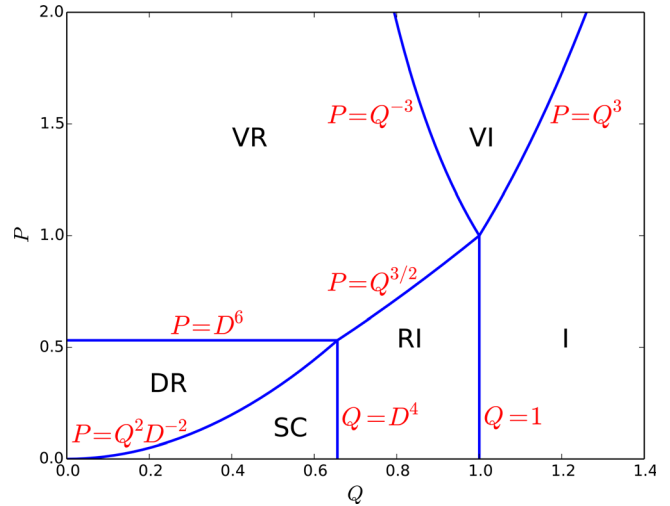


FIG. 1. Linear resonant plasma response regimes in Q - P space for the case $D = 0.9$. The various regimes are the diffusive-resistive (DR), the semi-collisional (SC), the resistive-inertial (RI), the viscous-resistive (VR), the viscous-inertial (VI), and the inertial (I).

where T_0 and χ_\perp are the electron/ion temperatures and the particle energy diffusivities at the resonant surface. It can be seen, by comparison with Figs. 1–3 in Ref. 7, that adopting a more realistic ordering for the perpendicular particle diffusivity, D_\perp , leads to a significant modification to the system of response regimes. We can identify the resistive-inertial, viscous-resistive, semi-collisional, diffusive-resistive, inertial, and viscous-inertial response regimes, found in this paper, with the RIi, VRii, SCii, HRii, I, and VI response regimes found in Ref. 7. Some of the response regimes found in Ref. 7—namely, the RIi, VRi, SCi, and HRi—no longer exist. On the other hand, we have found one new

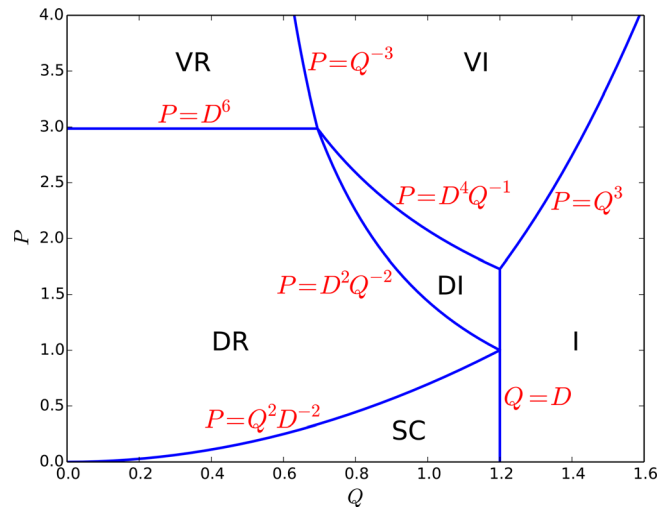


FIG. 2. Linear resonant plasma response regimes in Q - P space for the case $D = 1.2$. The various regimes are the diffusive-resistive (DR), the semi-collisional (SC), the diffusive-inertial (DI), the viscous-resistive (VR), the viscous-inertial (VI), and the inertial (I).

response regime—namely, the diffusive-inertial. On the whole, the system of response regimes shown in Figs. 1 and 2 is much simpler than that shown in Figs. 1–3 of Ref. 7.

Let $\hat{\delta}_* \sim p_*^{-1}$ be the normalized radial thickness of the resonant layer. Of course, the true thickness is $\delta = S^{-1/3} \hat{\delta}_* r_s$. It follows from Eq. (35) that the relative change in the perturbed helical flux, $\tilde{\psi}$, across the layer is

$$\left. \frac{d \ln \tilde{\psi}}{dX} \right|_{-\hat{\delta}/2}^{+\hat{\delta}/2} = \hat{\Delta} \hat{\delta}_* \sim \hat{\Delta} p_*^{-1}. \quad (76)$$

According to the analysis of Sec. V D, $\hat{\Delta} p_*^{-1}$ takes the respective values $Q^{1/2}$, $Q P^{1/3}$, Q^2/D^2 , and $Q P^{1/2}/D$ in the resistive-inertial, viscous-resistive, semi-collisional, and diffusive-resistive response regimes. Moreover, it is clear from Figs. 1 and 2 that these values are all much less than unity. In other words, it is indeed the case that $\tilde{\psi}$ does not vary substantially across a “constant- ψ ” resonant layer. On the other hand, according to the analysis of Sec. V E, $\hat{\Delta} p_*^{-1} \sim 1$ in the inertial, viscous-inertial, and diffusive-inertial response regimes, which implies that $\tilde{\psi}$ does vary substantially across a “nonconstant- ψ ” layer.

VI. LINEAR TEARING MODE STABILITY

A. Introduction

The aim of this section is to combine the analysis of Secs. II B and V to determine the linear stability of tearing modes in larger aspect-ratio tokamak plasmas. In this study, we shall assume that there is no externally generated, resonant magnetic perturbation (i.e., $\Psi_v = 0$).

B. Linear dispersion relation

Let us write $\omega = i\gamma + \omega_r$, where γ is the growth-rate of the tearing mode, whereas ω_r is the real frequency of the mode in the laboratory frame. We shall assume that $|\gamma| \ll |\omega_r|$. This assumption can easily be verified *a posteriori*. Asymptotic matching between the solution in the outer region and that in the thin linear layer surrounding the resonant surface yields a linear tearing mode dispersion relation of the form

$$S^{1/3} \hat{\Delta}(\gamma, \omega_r) = \Delta'_s, \quad (77)$$

where use has been made of Eqs. (3) and (36). Here, Δ'_s is the real tearing stability index, whereas the complex layer matching parameter, $\hat{\Delta}$, is defined in Eq. (35).

In a conventional tokamak plasma, $\Delta'_s \sim \mathcal{O}(1)$, and $S^{1/3} \gg 1$. (Here, we are neglecting $m = 1$ modes, which are not really tearing modes.) Hence, the previous dispersion relation can only be satisfied if ω_r takes a value that renders $|\hat{\Delta}| \ll 1$. It is clear from Eqs. (57), (58), (60), and (62) that this goal can be achieved in all of the constant- ψ linear response regimes if $\omega_r = \omega_{\perp e}$, where

$$\omega_{\perp e} = \omega_E + (1 + \lambda_e) \omega_{*e}. \quad (78)$$

We conclude that the tearing mode co-rotates with the electron fluid at the resonant surface.⁴ [In fact, the mode rotates slightly faster than the electron fluid because of the influence of the thermal force, which generates the additional factor λ_e in Eq. (78).]

C. Determination of linear growth-rates

Reusing the analysis of Secs. V C and V D, let us again suppose that there are two layers in p space (i.e., Fourier space). The small- p

layer turns out to be of width $|\hat{\gamma}|^{1/2}$, where $\hat{\gamma} = S^{1/3} \gamma \tau_H$. Given that we are effectively assuming that $|\hat{\gamma}| \ll 1$, the condition for the separation of the layer solution into two layers (i.e., that the width of the small- p layer is less than that of the large- p layer) is always satisfied. The large- p layer is governed by the equation

$$\frac{d^2 Y_e}{dp^2} - \frac{A(p)}{\tilde{B}(p)} Y_e = 0, \quad (79)$$

where

$$A(p) = (1 + \lambda_e) \tau_e (1 + \lambda_e \tau_e) Q_*^2 - i(1 + \lambda_e \tau_e) Q_* (P_\phi + P_\perp) p^2 + P_\phi P_\perp p^4, \quad (80)$$

$$\tilde{B}(p) = P_\perp - i(1 + \lambda_e)(1 + \lambda_e \tau_e) Q_* D^2 + \tau_e (1 + \lambda_e \tau_e) D^2 P_\phi p^2. \quad (81)$$

Here, $Q_* = S^{1/3} \omega_* \tau_H$, where $\omega_* = -(m/r_s) (1/e n_0 B_z) (dp/dr)_s$. In the following, ω_* is assumed to be positive. The boundary conditions on Eq. (79) are that Y_e is bounded as $p \rightarrow \infty$, and

$$Y_e(p) = Y_0 \left[1 - \frac{\hat{\Delta} p}{\pi \hat{\gamma}} + \mathcal{O}(p^2) \right], \quad (82)$$

as $p \rightarrow 0$.

In the various constant- ψ linear growth-rate regimes considered in this section, Eq. (79) reduces to an equation of the form $d^2 Y_e/dp^2 - C p^m Y_e = 0$, where m is real and non-negative, and C is a complex constant. As described in Sec. V C, the solution of this equation that is bounded as $p \rightarrow \infty$ can be matched to the small- p asymptotic form (82) to give

$$\hat{\Delta} = \pi \hat{\gamma} \frac{\Gamma(1 - \nu)}{\Gamma(\nu)} \nu^{2\nu-1} C^\nu, \quad (83)$$

where $\nu = 1/(m + 2)$. The width of the large- p layer in p space is $C^{-\nu}$.

As before, we shall assume that $P_\phi \sim P_\perp \sim P$, $\lambda_e \sim \mathcal{O}(1)$, and $\tau \sim \mathcal{O}(1)$, for the sake of simplicity. Suppose that $Q_* \gg P p^2$ and $P \gg Q_* D^2$. It follows that $\nu = 1/2$ and

$$C = \tau_e (1 + \lambda_e)(1 + \lambda_e \tau_e) \frac{Q_*^2}{P_\perp}. \quad (84)$$

Hence,

$$\hat{\Delta} = \pi \hat{\gamma} [\tau_e (1 + \lambda_e)(1 + \lambda_e \tau_e)]^{1/2} \frac{Q_*}{P_\perp^{1/2}}, \quad (85)$$

and $p_* \sim P^{1/2}/Q_*$. This resistive-inertial growth-rate regime is valid when $P \ll Q_*^{3/2}$ and $P \gg Q_* D^2$. Making use of Eq. (77), the corresponding tearing mode growth-rate is

$$\gamma = \frac{\hat{\Delta}'_s}{\pi [\tau_e (1 + \lambda_e)(1 + \lambda_e \tau_e)]^{1/2}} \frac{1}{\omega_* \tau_H \tau_R^{1/2} \tau_\perp^{1/2}}. \quad (86)$$

Note that this growth-rate is significantly different from the classical resistive-inertial growth-rate derived in Ref. 11, as a consequence of finite perpendicular particle diffusivity.

Suppose that $Q_* \ll P p^2$ and $D^2 p^2 \ll 1$. It follows that $\nu = 1/6$ and $C = P_\phi$. Hence,

$$\hat{\Delta} = 6^{2/3} \pi \hat{\gamma} \frac{\Gamma(5/6)}{\Gamma(1/6)} P^{1/6}, \tag{87}$$

and $p_* \sim P^{-1/6}$. This viscous-resistive growth-rate regime is valid when $P \gg Q_*^{3/2}$ and $P \gg D^6$. The corresponding tearing mode growth-rate is

$$\gamma = \frac{\Delta'_s}{6^{2/3} \pi [\Gamma(5/6)/\Gamma(1/6)]} \frac{\tau_\phi^{1/6}}{\tau_H^{1/3} \tau_R^{5/6}}. \tag{88}$$

Suppose that $Q_* \gg P p^2$ and $P \ll Q_* D^2$. It follows that $\nu = 1/2$ and

$$C = i \tau_e \frac{Q_*}{D^2}. \tag{89}$$

Hence,

$$\hat{\Delta} = e^{i\pi/4} \pi \hat{\gamma} \tau_e^{1/2} \frac{Q_*^{1/2}}{D}, \tag{90}$$

and $p_* \sim D/Q_*^{1/2}$. This semi-collisional growth-rate regime is valid when $P \ll Q_*^2/D^2$ and $P \ll Q_* D^2$. The corresponding tearing mode growth-rate is

$$\gamma = \frac{\hat{\Delta}'_s}{\sqrt{2} \pi} \frac{d_\beta/r_s}{\omega_*^{1/2} \tau_H \tau_R^{1/2}}. \tag{91}$$

Here, we have neglected the imaginary component of γ because it merely gives rise to a small correction to the real frequency of the mode.

Suppose, finally, that $Q_* \ll P p^2$ and $D^2 p^2 \gg 1$. It follows that $\nu = 1/4$ and

$$C = \frac{P_\perp \tau_e}{(1 + \lambda_e \tau_e) D^2}. \tag{92}$$

Hence,

$$\hat{\Delta} = 2\pi \hat{\gamma} \frac{\Gamma(3/4)}{\Gamma(1/4)} \frac{\tau_e^{1/4} P_\perp^{1/4}}{(1 + \lambda_e \tau_e)^{1/4} D^{1/2}}, \tag{93}$$

and $p_* \sim D^{1/2}/P^{1/4}$. This diffusive-resistive growth-rate regime is valid when $P \gg Q_*^2/D^2$ and $P \ll D^6$. The corresponding tearing mode growth-rate is

$$\gamma = \frac{\Delta'_s}{2\pi [\Gamma(3/4)/\Gamma(1/4)]} \frac{(d_\beta/r_s)^{1/2} \tau_\perp^{1/4}}{(1 + \lambda_e \tau_e)^{-1/4} \tau_H^{1/2} \tau_R^{3/4}}. \tag{94}$$

D. Discussion

According to Eqs. (86), (88), (91), and (94), a linear tearing mode is unstable when the tearing stability index, Δ'_s , is positive, and is stable otherwise.¹¹ Moreover, the perturbed magnetic field associated with the mode corotates with the electron fluid at the resonant surface.⁴

There are three main factors that affect the growth-rate of a tearing mode in a conventional tokamak plasma: first, the strength of diamagnetic flows in the plasma, which is parameterized by the diamagnetic frequency, ω_* (and by the normalized diamagnetic frequency, Q_*); second, the anomalous perpendicular diffusion of

momentum and particles, which is parameterized by the momentum and particle confinement timescales, τ_ϕ and τ_\perp (and by the magnetic Prandtl number, P); and third, finite ion sound radius effects, which are parameterized by the ion sound radius, d_β (and by the normalized ion sound radius D).

There are four tearing-mode growth-rate regimes—the resistive-inertial, the viscous-resistive, the semi-collisional, and the diffusive-resistive—and their extents in Q_* - P space are illustrated in Fig. 3. The value of Q_* can be estimated from Eq. (73), with ω replaced by ω_* . Note that Fig. 3 differs somewhat from Figs. 1 and 2 because in the latter two figures it is assumed that $|Q - Q_E - (1 + \lambda_e) Q_e| \sim |Q - Q_E| \sim |Q - Q_E - Q_i|$ whereas in the former figure, it is assumed that $|Q - Q_E - (1 + \lambda_e) Q_e| \ll |Q - Q_E| \sim |Q - Q_E - Q_i|$. This refined ordering eliminates the nonconstant- ψ response regimes and significantly modifies the extent of the resistive-inertial response regime.

The absence of nonconstant- ψ response regimes in Fig. 3 should come as no surprise. As we saw in Sec. V F, nonconstant- ψ resonant layers are characterized by $\hat{\Delta} \sim \hat{\delta}_*^{-1} = S^{-1/3} (r_s/\delta)$, where δ is the layer thickness. Hence, according to Eq. (77), asymptotic matching of such a layer to the outer solution is only possible if $|\Delta'_s| \sim r_s/\delta \gg 1$ (given that resonant layers in tokamak plasmas are invariably very thin compared to the minor radius of the plasma). However, low- m tearing modes in conventional tokamak plasmas are characterized by $|\Delta'_s| \sim \mathcal{O}(1)$ rather than $|\Delta'_s| \gg 1$.

Diamagnetic flows modify the tearing mode growth-rate in the resistive-inertial and semi-collisional growth-rate regimes, but not in the diffusive-resistive and viscous-resistive regimes. Anomalous perpendicular transport modifies the growth-rate in all regimes except the semi-collisional regime. Finally, finite ion sound radius effects modify the growth-rate in the diffusive-resistive and semi-collisional regimes, but not in the other two regimes.

Increasing the diamagnetic flow strength decreases the tearing mode growth-rate such that it eventually scales as $\omega_*^{-1/2}$. Increasing the anomalous perpendicular transport increases the growth-rate such

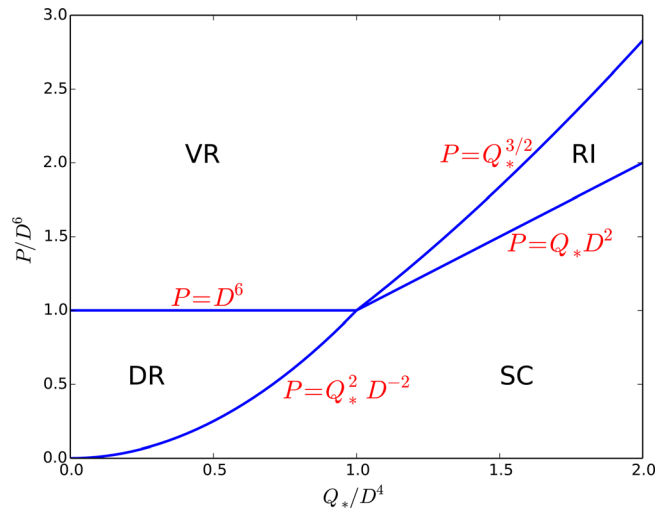


FIG. 3. Linear tearing mode growth-rate regimes in Q_* - P space. The various regimes are the diffusive-resistive (DR), the semi-collisional (SC), the viscous-resistive (VR), and the resistive-inertial (RI).

that it eventually scales as $P_\phi^{1/6}$. Increasing the ion sound radius increases the tearing mode growth-rate, but the increase eventually saturates (i.e., the growth-rate ends up scaling as d_β^0).

Finally, it is clear from Eq. (77) that the layer solution can only be matched to the outer solution when ω_r takes a value that renders $|\hat{\Delta}| \ll 1$. We have seen that this is possible in all constant- ψ response regimes when $\omega_r = \omega_E + (1 + \lambda_e) \omega_{*e}$ (i.e., when the tearing perturbation corotates with the electron fluid at the resonant surface.) However, Eq. (57) seems to suggest that in the resistive-inertial regime it might be possible to successfully match the layer solution to the outer solution when $\omega_r = \omega_E$ (i.e., $Q = Q_E$), or when $\omega_r = \omega_E + \omega_{*i}$ (i.e., $Q = Q_E + Q_i$), giving tearing mode perturbations that corotate with the MHD fluid and the ion fluid, respectively, at the resonant surface. In fact, we can see that this is not the case, because if we set $\omega_r = \omega_E$ or $\omega_r = \omega_E + \omega_{*i}$ then Eq. (50) does not lead to a layer solution characterized by $|\hat{\Delta}| \ll 1$, as a consequence of finite anomalous perpendicular transport.¹² Likewise, Eq. (60) seems to suggest that in the semi-collisional response regime it might be possible to successfully match the layer solution to the outer solution when $\omega_r = \omega_E$, giving rise to a tearing perturbation that corotates with the MHD fluid at the resonant surface. In fact, this possibility is also ruled out by finite anomalous perpendicular transport. To be more exact, the spurious resonances at $\omega_r = \omega_E$ and $\omega_r = \omega_E + \omega_{*i}$ originate from the fact that, in the absence of finite perpendicular transport (i.e., $P_\phi = P_\perp = 0$), the function $A(p)$, specified in Eq. (45), takes the value zero when $Q = Q_E + Q_i$ or $Q = Q_E$. However, in the presence of finite perpendicular transport, the function remains finite at $Q = Q_E + Q_i$ and $Q = Q_E$, and the spurious resonances are eliminated.

VII. ERROR-FIELD PENETRATION

A. Introduction

Tokamak plasmas are invariably subject to small amplitude, static, resonant magnetic perturbations—known as error-fields—which are primarily generated by field-coil misalignments and uncompensated coil feeds. Error-fields can drive magnetic reconnection, resulting in the formation of locked (i.e., non-rotating in the laboratory frame) magnetic island chains, in intrinsically tearing-stable plasmas. Tokamak plasmas containing locked magnetic island chains often terminate in disruptions.² Fortunately, error-field driven magnetic reconnection is strongly suppressed by the naturally occurring rotation of such plasmas. However, when the error-field amplitude rises above a certain critical amplitude, the rotation at the resonant surface is suddenly arrested, and error-field driven reconnection proceeds unhindered. This phenomenon is known as error-field penetration.^{10,13} The scenario just outlined has been observed in a number of tokamak experiments.^{16–24}

In this section, we shall calculate the critical error-field amplitude required to trigger penetration on the assumption that, prior to penetration, the rotational suppression of error-field driven magnetic reconnection is sufficiently strong that the resonant plasma response is governed by linear layer physics.^{7,10,13}

B. Asymptotic matching

It is convenient to parametrize the error-field in terms of the vacuum magnetic flux, Ψ_v , that it generates at the resonant surface. Equations (3) and (36) can be combined to give

$$\Psi_s = \frac{\Psi_v}{1 + \Delta/(-\Delta'_s)}, \quad (95)$$

where $\Delta = S^{1/3} \hat{\Delta}$. The previous equation specifies the reconnected magnetic flux, Ψ_s , driven at the resonant surface by the error-field. The complex layer parameter, Δ , specifies the strength of a shielding current that is induced in the resonant layer, and acts to prevent driven magnetic reconnection. Note that $\Delta'_s < 0$, because the plasma is assumed to be intrinsically tearing stable.

C. Resonant layer response

Generally speaking, we do not expect the plasma flow at the resonant surface to be sufficiently large to cause a breakdown of the constant- ψ approximation. We also expect $P_\phi, P_\perp \gg 1$. In other words, we expect the resistive evolution timescale to significantly exceed both the momentum and energy/particle confinement timescales (which it generally does by at least, a factor of 10). Hence, consulting Figs. 1 and 2, we can see that the appropriate linear resonant response regime is either the viscous-resistive regime or the diffusive-resistive regime.

According to the analysis of Sec. V D, and making use of the fact that $\omega = Q = 0$ for a static perturbation, in the viscous-resistive response regime we have

$$\Delta_{VR} = i \tau_{VR} (\omega_{\perp e} + \Delta \omega_E), \quad (96)$$

where $\tau_{VR} = \tau_R \hat{\delta}_{VR}$, and

$$\hat{\delta}_{VR} = \frac{6^{2/3} \pi \Gamma(5/6)}{\Gamma(1/6)} \frac{\tau_H^{1/3}}{\tau_R^{1/6} \tau_\phi^{1/6}}, \quad (97)$$

is the viscous-resistive layer width (normalized to r_s). Here, $\omega_{\perp e}$ is defined in Eq. (78), and $\Delta \omega_E$ is the modification to the $\mathbf{E} \times \mathbf{B}$ frequency generated at the resonant surface by the electromagnetic torques that develop there in response to the error-field. Likewise, in the diffusive-resistive regime we have

$$\Delta_{DR} = i \tau_{DR} (\omega_{\perp e} + \Delta \omega_E), \quad (98)$$

where $\tau_{DR} = \tau_R \hat{\delta}_{DR}$, and

$$\hat{\delta}_{DR} = \frac{2\pi \Gamma(3/4)}{\Gamma(1/4)} \frac{\tau_H^{1/2}}{(d_\beta/r_s)^{1/2} \tau_R^{1/4} \tau_\perp^{1/4}}, \quad (99)$$

is the diffusive-resistive layer width (normalized to r_s).

We can combine the previous two results to give the composite layer response equation

$$\Delta = i \tau_s (\omega_{s0} + \Delta \omega_E), \quad (100)$$

where $\tau_s = \tau_R \hat{\delta}$,

$$\hat{\delta} = \frac{\hat{\delta}_{VR} \hat{\delta}_{DR}}{\hat{\delta}_{VR} + \hat{\delta}_{DR}}, \quad (101)$$

and $\omega_{s0} = \omega_{\perp e}$. Here, $\omega_{\perp e}$ is understood to take its unperturbed (by any electromagnetic torques that develop in the resonant layer) value.

According to Eq. (100), the resonant layer responds to the error-field perturbation in an analogous manner to a thin, rigid, resistive wall whose resistivity matches that of the plasma at the resonant surface, and whose radial thickness is $\delta = r_s \hat{\delta}$. Note, however, that the

effective wall corotates with the electron fluid at the resonant surface. Moreover, it is the rotation of the effective wall that generates the shielding current.

Equations (3), (95), and (100) yield

$$\Delta\Psi_s = i\tau_s(\omega_{s0} + \Delta\omega_E)\Psi_s, \quad (102)$$

and

$$\Psi_s = \frac{\Psi_v}{1 + i\tau_s(\omega_{s0} + \Delta\omega_E)/(-\Delta'_s)}. \quad (103)$$

Now, in a conventional Ohmically heated tokamak plasma, $\tau_s\omega_{s0} \gg (-\Delta'_s)$. Consequently, according to the previous equation, the shielding current that develops at the resonant surface, in response to the local plasma rotation, is strong enough to largely suppress error-field-driven magnetic reconnection (i.e., $|\Psi_s| \ll |\Psi_v|$). However, the shielding current also gives rise to a localized electromagnetic locking torque at the resonant surface that acts to slow the plasma rotation.

D. Torque balance

According to the analysis of Ref. 25 (see, in particular, Appendix C), the shift in the $\mathbf{E} \times \mathbf{B}$ frequency at the resonant surface induced by the error-field locking torque can be written,

$$\Delta\omega_E = -\sum_{p=1,\infty} (\alpha_p + \beta_p), \quad (104)$$

where

$$\alpha_p = \frac{(r_s/a)^2 [J_1(j_{1p} r_s/a)]^2}{[J_2(j_{1p})]^2 [1 + (\tau_\theta/\tau_\phi)(r_s/a)^2 j_{1p}^2]} \left(\frac{\tau_\theta}{\tau_H^2}\right) \times \text{Im}\left(\frac{\Delta\Psi_s}{\Psi_s}\right) |\Psi_s|^2 \left(\frac{L_s}{B_z r_s^2}\right)^2, \quad (105)$$

$$\beta_p = \left(\frac{\epsilon_s}{q_s}\right)^2 \frac{[J_0(j_{0p} r_s/a)]^2}{[J_1(j_{0p})]^2 j_{0p}^2} \left(\frac{\tau_\phi}{\tau_H^2}\right) \text{Im}\left(\frac{\Delta\Psi_s}{\Psi_s}\right) |\Psi_s|^2 \left(\frac{L_s}{B_z r_s^2}\right)^2. \quad (106)$$

Here, τ_θ is the neoclassical poloidal flow-damping time evaluated at the resonant surface, $J_m(z)$ is a Bessel function, and j_{mp} denotes the p th zero of $J_m(z)$. Making use of Eqs. (102) and (100), and the previous two equations, we obtain

$$\alpha_p = \frac{(r_s/a)^2 [J_1(j_{1p} r_s/a)]^2}{[J_2(j_{1p})]^2 [1 + (\tau_\theta/\tau_\phi)(r_s/a)^2 j_{1p}^2]} \left(\frac{\tau_\theta}{\tau_H^2}\right) \times \frac{\tau_s(\omega_{s0} + \Delta\omega_E)}{1 + \tau_s^2(\omega_{s0} + \Delta\omega_E)^2/(-\Delta'_s)^2} \left(\frac{w_v}{r_s}\right)^4, \quad (107)$$

$$\beta_p = \left(\frac{\epsilon_s}{q_s}\right)^2 \frac{[J_0(j_{0p} r_s/a)]^2}{[J_1(j_{0p})]^2 j_{0p}^2} \left(\frac{\tau_\phi}{\tau_H^2}\right) \frac{\tau_s(\omega_{s0} + \Delta\omega_E)}{1 + \tau_s^2(\omega_{s0} + \Delta\omega_E)^2/(-\Delta'_s)^2} \left(\frac{w_v}{r_s}\right)^4, \quad (108)$$

where

$$w_v = \left(\frac{L_s |\Psi_v|}{B_z}\right)^{1/2} \quad (109)$$

is one-quarter of the full width of the vacuum magnetic island chain that the error-field would drive at the resonant surface where there is no shielding current. In Eq. (104), the term $\sum_{p=1,\infty} (-\alpha_p)$ denotes the shift in the $\mathbf{E} \times \mathbf{B}$ frequency at the resonant surface that is induced by changes in the poloidal ion fluid velocity, whereas $\sum_{p=1,\infty} (-\beta_p)$ denotes the shift that induced by changes in the toroidal ion fluid velocity.

Making use of the identities (which have been verified numerically)

$$\lim_{\epsilon \rightarrow 0} \sum_{p=1,\infty} \frac{\sqrt{\epsilon} [J_1(j_{1p} x)]^2}{[J_2(j_{1p})]^2 (1 + \epsilon j_{1p}^2)} = \frac{1}{4x}, \quad (110)$$

$$\sum_{p=1,\infty} \frac{[J_0(j_{0p} x)]^2}{[J_1(j_{0p} x)]^2 j_{0p}^2} = \frac{1}{2} \ln\left(\frac{1}{x}\right), \quad (111)$$

Equations (104), (107), and (108) yield

$$\Delta\omega_E = -\frac{1}{4} \left(\frac{\epsilon_s}{q_s}\right)^2 \left(\frac{\tau_V}{\tau_H^2}\right) \frac{\tau_s(\omega_{s0} + \Delta\omega_E)}{1 + \tau_s^2(\omega_{s0} + \Delta\omega_E)^2/(-\Delta'_s)^2} \left(\frac{w_v}{r_s}\right)^4, \quad (112)$$

where

$$\tau_V = \left(\frac{q_s}{\epsilon_s}\right)^2 (\tau_\theta \tau_\phi)^{1/2} + 2 \ln\left(\frac{a}{r_s}\right) \tau_\phi. \quad (113)$$

Here, we have assumed that $\tau_\theta \ll \tau_\phi$ (as is invariably the case in tokamak plasmas).

The torque balance equation, (112), can be written in the nondimensional form

$$T_{VS}(v) = T_{EM}(v), \quad (114)$$

where

$$v = \frac{\omega_{s0} + \Delta\omega_E}{\omega_{s0}}, \quad (115)$$

$$T_{VS}(v) = 1 - v, \quad (116)$$

$$T_{EM}(v) = \frac{1}{4} \frac{v}{\zeta_s^2 + v^2} \left(\frac{w_v}{w_{\text{crit}}}\right)^4, \quad (117)$$

$$\zeta_s = \frac{(-\Delta'_s)}{\tau_s \omega_{s0}} \ll 1, \quad (118)$$

$$\frac{w_{\text{crit}}}{r_s} = \left[\frac{1}{(-\Delta'_s)} \left(\frac{q_s}{\epsilon_s}\right) \left(\frac{\tau_s}{\tau_V}\right)^{1/2} (\omega_{s0} \tau_H) \right]^{1/2}. \quad (119)$$

In Eq. (114), the dimensionless parameter v takes the value unity when the plasma (i.e., electron fluid) rotation at the resonant surface is unmodified by the electromagnetic locking torque, and zero when the electromagnetic locking torque has reduced the rotation to zero. The electromagnetic locking torque itself is represented by the function $T_{EM}(v)$, whereas the viscous restoring torque that acts to oppose changes in the plasma rotation is represented by the function $T_{VS}(v)$. Note that the viscous restoring torque varies linearly with v , whereas the electromagnetic locking torque varies with v in a nonmonotonic manner.

The solutions of the torque balance equation (114) correspond to the crossing points of the viscous and electromagnetic torque curves. If reduced vacuum island width, w_v , lies below the critical value w_{crit}

then there are three crossing points. However, the intermediate crossing point corresponds to a dynamically unstable equilibrium. Hence, we deduce that there are two physical solution branches of the torque balance equation. The large- v solution branch corresponds to so-called shielded solutions in which the plasma rotation at the resonant surface is large enough to strongly suppress driven magnetic reconnection. The small- v solution branch corresponds to so-called unshielded solutions in which the plasma rotation at the resonant surface is too weak to suppress driven magnetic reconnection.^{10,13} On the shielded solution branch, Eq. (114) possesses the approximate solution,

$$v \simeq \frac{1}{2} \left(1 - \left[1 - \left(\frac{w_v}{w_{\text{crit}}} \right)^4 \right]^{1/2} \right). \quad (120)$$

It can be seen that as the vacuum island width increases, the plasma rotation at the resonant surface decreases. However, when w_v attains the critical value w_{crit} , and the rotation is reduced to half of its original value, the shielded solution branch ceases to exist, and there is a bifurcation to the unshielded solution branch.^{10,13} The bifurcation is associated with a sudden decrease in the plasma rotation at the resonant surface, and a sudden increase in the driven reconnected magnetic flux, and is identified with the error-field penetration phenomenon observed in tokamak experiments.

It is convenient to parameterize the critical error-field required to trigger penetration in terms of the critical vacuum radial magnetic field, b_r , that must be driven at the resonant surface. It is easily demonstrated that

$$\left(\frac{b_{\text{rcrit}}}{B_z} \right) = (n \epsilon_s s_s) \left(\frac{w_{\text{rcrit}}}{r_s} \right)^2, \quad (121)$$

which implies that

$$\left(\frac{b_{\text{rcrit}}}{B_z} \right) = \frac{m s_s}{(-\Delta'_s)} \left(\frac{\tau_s}{\tau_V} \right)^{1/2} (\omega_{s0} \tau_H). \quad (122)$$

Note that the critical radial field needed to trigger error-field penetration increases linearly with the unperturbed plasma rotation at the resonant surface, which is parameterized by ω_{s0} .

E. Scaling analysis

The aim of this section is to use the previous analysis to predict the scaling of the error-field penetration threshold with the standard dimensionless scaling parameters,²⁶

$$\rho_* = \frac{T_0^{1/2} m_i^{1/2}}{e B_z L_s} = 3.23 \times 10^{-3} \frac{T_0^{1/2}(\text{keV})}{B_z(\text{T}) L_s(\text{m})}, \quad (123)$$

$$\nu_* = \frac{L_s n_0 e^2 \eta_{\parallel}}{m_e^{1/2} T_0^{1/2}} = 5.30 \times 10^{-4} \frac{L_s(\text{m}) n_0 (10^{19} \text{m}^{-3})}{T_0^2(\text{keV})}, \quad (124)$$

$$\beta_* = \frac{\mu_0 n_0 T_0}{B_z^2} = 2.01 \times 10^{-3} \frac{n_0 (10^{19} \text{m}^{-3}) T_0(\text{keV})}{B_z^2(\text{T})}, \quad (125)$$

where $T_0 = p_0/n_0$. In the following, for the sake of simplicity, we shall assume that $T_i = T_e = T_0$, $\omega_{s0} = \omega_{*s}$, and $-(d \ln p/dr)_{r_s} = 1/r_s$. We shall also neglect $\mathcal{O}(1)$ constants, such as m , s_s , η_e , η_{\parallel} , λ_e , τ , π , $\ln(r_s/a)$, etc.

We can write

$$\frac{1}{\tau_{\theta}} = \left(\frac{q_s}{\epsilon_s} \right)^2 \left(\frac{\mu_{00}^i}{\tau_i} \right), \quad (126)$$

where τ_i is the Braginskii ion collision time, and μ_{00}^i is a dimensionless neoclassical viscosity coefficient defined in Ref. 25. Both quantities are evaluated at the resonant surface.

In a tokamak plasma, the energy confinement timescale, τ_E , satisfies the constraint

$$\frac{n_0 T_0}{\tau_E} = \eta_{\parallel} \left(\frac{B_z}{\mu_0 L_s} \right)^2 \mathcal{P}, \quad (127)$$

which is obtained by equating the energy loss rate to the heating rate. Here, \mathcal{P} is the ratio of the total heating rate to the Ohmic heating rate. (Thus, $\mathcal{P} = 1$ in an Ohmically heated tokamak.) Let us assume that the momentum confinement time, τ_{ϕ} , and the particle/energy confinement time, τ_{\perp} , are both equal to τ_E , as is generally (approximately) the case in tokamak plasmas.²⁷

It is easily demonstrated that

$$\omega_{s0} \tau_H \simeq \left(\frac{q_s}{\epsilon_s} \right)^2 \rho_* \beta_*^{1/2}, \quad (128)$$

$$\frac{\tau_H}{\tau_R} \simeq \left(\frac{q_s}{\epsilon_s} \right)^2 \left(\frac{m_e}{m_i} \right)^{1/2} \rho_*^2 \nu_* \beta_*^{-1/2}, \quad (129)$$

$$\frac{\tau_{\phi}}{\tau_R} = \frac{\tau_{\perp}}{\tau_R} \simeq \left(\frac{q_s}{\epsilon_s} \right)^2 \beta_* \mathcal{P}^{-1}, \quad (130)$$

$$\frac{\tau_i}{\tau_R} \simeq \left(\frac{q_s}{\epsilon_s} \right)^2 \left(\frac{m_e}{m_i} \right)^{1/2} \rho_*^2 \beta_*^{-1}, \quad (131)$$

$$\frac{d_{\beta}}{r_s} \simeq \left(\frac{q_s}{\epsilon_s} \right) \rho_*. \quad (132)$$

Hence, we deduce that

$$\begin{aligned} \hat{\delta}_{\text{VR}} &= \left(\frac{\tau_{\text{VR}}}{\tau_R} \right) \simeq \left(\frac{\tau_H}{\tau_R} \right)^{1/3} \left(\frac{\tau_R}{\tau_{\phi}} \right)^{1/6} \\ &\simeq \left(\frac{q_s}{\epsilon_s} \right)^{1/3} \left(\frac{m_e}{m_i} \right)^{1/6} \rho_*^{2/3} \nu_*^{1/3} \beta_*^{-1/3} \mathcal{P}^{1/6}, \end{aligned} \quad (133)$$

$$\begin{aligned} \hat{\delta}_{\text{DR}} &= \left(\frac{\tau_{\text{DR}}}{\tau_R} \right) \simeq \left(\frac{\tau_H}{\tau_R} \right)^{1/2} \left(\frac{\tau_R}{\tau_{\perp}} \right)^{1/4} \left(\frac{r_s}{d_{\beta}} \right)^{1/2} \\ &\simeq \left(\frac{m_e}{m_i} \right)^{1/4} \rho_*^{1/2} \nu_*^{1/2} \beta_*^{-1/2} \mathcal{P}^{1/4}, \end{aligned} \quad (134)$$

$$\frac{\sqrt{\tau_{\phi} \tau_i / \mu_{00}^i}}{\tau_R} \simeq \left(\frac{q_s}{\epsilon_s} \right)^2 \left(\frac{m_e}{m_i} \right)^{1/4} \rho_* \mathcal{P}^{-1/2}. \quad (135)$$

Here, we have treated μ_{00}^i as an ignorable $\mathcal{O}(1)$ constant that is independent of ρ_* , ν_* , and β_* (as is indeed the case provided that the plasma at the resonant surface lies in the banana collisionality regime).²⁸

There are four error-field penetration regimes, depending on whether the plasma response in the resonant layer lies in the viscous-resistive or the diffusive-resistive regime, and depending on whether the change in the $\mathbf{E} \times \mathbf{B}$ frequency at the resonant surface, induced by

the electromagnetic locking torque, is predominately due to a toroidal or a poloidal shift in the local ion fluid velocity. The plasma response in the resonant layer is in the viscous-resistive, rather than the diffusive-resistive, regime when $\tau_{VR} < \tau_{DR}$, or

$$\rho_* < \left(\frac{\epsilon_s}{q_s}\right)^2 \left(\frac{m_e}{m_i}\right)^{1/2} \nu_* \beta_*^{-1} \mathcal{P}^{1/2}, \quad (136)$$

and vice versa. The change in the $\mathbf{E} \times \mathbf{B}$ frequency at the resonant surface is predominately due to a toroidal, rather than poloidal, ion fluid velocity shift when

$$\rho_* < \left(\frac{\epsilon_s}{q_s}\right) \left(\frac{m_i}{m_e}\right)^{1/4} \beta_* \mathcal{P}^{-1/2}, \quad (137)$$

and vice versa.

Let us define

$$\begin{aligned} \hat{\rho}_* &= \left(\frac{q_s}{\epsilon_s}\right) \left(\frac{m_e}{m_i}\right)^{1/4} \rho_* \mathcal{P}^{1/2} \\ &= 0.245 \left(\frac{B_z}{B_\theta}\right) \mathcal{P}^{1/2} \frac{B_z(\text{T})}{L_s(\text{m}) n_0 (10^{19} \text{m}^{-3}) T_0^{1/2}(\text{keV})}, \end{aligned} \quad (138)$$

$$\hat{\nu}_* = \left(\frac{\epsilon_s}{q_s}\right) \left(\frac{m_e}{m_i}\right)^{3/4} \nu_* \mathcal{P} = 0.467 \left(\frac{B_\theta}{B_z}\right) \mathcal{P} \frac{L_s(\text{m}) B_z^4(\text{T})}{n_0 (10^{19} \text{m}^{-3}) T_0^4(\text{keV})}, \quad (139)$$

As is illustrated in Fig. 4, the viscous-resistive-toroidal regime (i.e., a viscous-resistive layer response combined with a predominately toroidal change in the ion fluid velocity) corresponds to $\hat{\rho}_* < \beta_*^{-1} \nu_*$ and $\hat{\rho}_* < \beta_*$, the viscous-resistive-poloidal regime corresponds to $\hat{\rho}_* < \beta_*^{-1} \nu_*$ and $\hat{\rho}_* > \beta_*$, the diffusive-resistive-poloidal regime corresponds to $\hat{\rho}_* > \beta_*^{-1} \nu_*$ and $\hat{\rho}_* > \beta_*$, and the diffusive-resistive-toroidal regime corresponds to $\hat{\rho}_* > \beta_*^{-1} \nu_*$ and $\hat{\rho}_* < \beta_*$. It can be seen that higher/lower plasma collisionality (i.e., ν_*) favors the

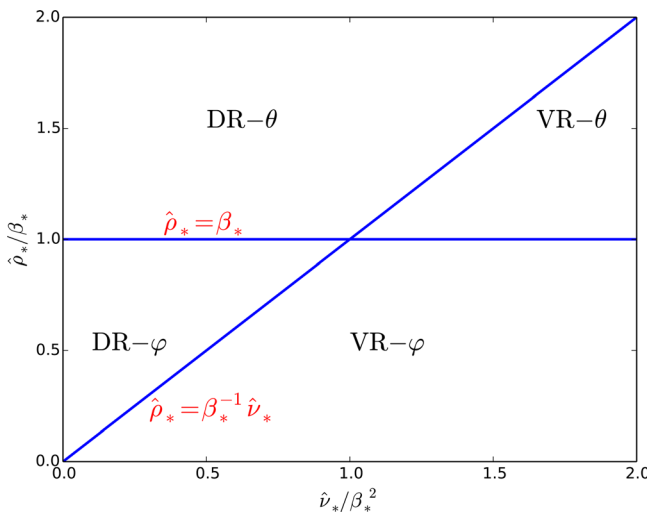


FIG. 4. Error-field penetration regimes in ν_* - $\hat{\rho}_*$ space. The various regimes are the viscous-resistive-toroidal (VR- ϕ), the viscous-resistive-poloidal (VR- θ), the diffusive-resistive-poloidal (DR- θ), and the diffusive-resistive-toroidal (DR- ϕ).

viscous-resistive/diffusive-resistive response regime, whereas a larger/smaller ion sound radius (i.e., ρ_*) favors poloidal/toroidal changes in the ion fluid velocity. Finally, higher/lower plasma pressure (i.e., β_*) favors the diffusive-resistive/viscous-resistive response regime and toroidal/poloidal changes in the ion fluid velocity.

In the viscous-resistive-toroidal response regime, the critical vacuum radial magnetic field at the resonant surface required to trigger error-field penetration is⁷

$$\left(\frac{b_{rcrit}}{B_z}\right)_{VR-\phi} \simeq \left(\frac{q_s}{\epsilon_s}\right)^{7/6} \left(\frac{m_e}{m_i}\right)^{1/12} \rho_*^{4/3} \nu_*^{1/6} \beta_*^{-1/6} \mathcal{P}^{7/12}, \quad (140)$$

where we have treated $-\Delta'_s$ as an ignorable $\mathcal{O}(1)$ parameter. Likewise, the critical field required to trigger penetration in the viscous-resistive-poloidal regime is

$$\left(\frac{b_{rcrit}}{B_z}\right)_{VR-\theta} \simeq \left(\frac{q_s}{\epsilon_s}\right)^{2/3} \left(\frac{m_e}{m_i}\right)^{-1/24} \rho_*^{5/6} \nu_*^{1/6} \beta_*^{1/3} \mathcal{P}^{1/3}. \quad (141)$$

The critical field required to trigger penetration in the diffusive-resistive-poloidal regime is

$$\left(\frac{b_{rcrit}}{B_z}\right)_{DR-\theta} \simeq \left(\frac{q_s}{\epsilon_s}\right)^{1/2} \left(\frac{m_e}{m_i}\right)^0 \rho_*^{3/4} \nu_*^{1/4} \beta_*^{1/4} \mathcal{P}^{3/8}. \quad (142)$$

Finally, the critical field required to trigger penetration in the diffusive-resistive-toroidal regime is

$$\left(\frac{b_{rcrit}}{B_z}\right)_{DR-\phi} \simeq \left(\frac{q_s}{\epsilon_s}\right) \left(\frac{m_e}{m_i}\right)^{1/8} \rho_*^{5/4} \nu_*^{1/4} \beta_*^{-1/4} \mathcal{P}^{5/8}. \quad (143)$$

Generally speaking, Ohmically heated tokamak plasmas all have similar β_* values. However, both ν_* and ρ_* are smaller in devices of larger physical size. Hence, Eqs. (140)–(143) imply that the error-field penetration threshold is smaller in larger devices.

Let n , T , B , and R represent the typical electron number density, plasma temperature, toroidal magnetic field-strength, and physical dimension (i.e., major radius) of an Ohmically heated tokamak. Using $\rho_* \sim T^{1/2} B^{-1} R^{-1}$, $\nu_* \sim n T^{-2} R$, and $\beta_* \sim n T B^{-2}$, we deduce that

$$\left(\frac{b_{rcrit}}{B_z}\right)_{VR-\phi} \propto n^0 T^{1/6} B^{-1} R^{-7/6} \mathcal{P}^{7/12}, \quad (144)$$

$$\left(\frac{b_{rcrit}}{B_z}\right)_{VR-\theta} \propto n^{1/2} T^{5/12} B^{-3/2} R^{-2/3} \mathcal{P}^{1/3}, \quad (145)$$

$$\left(\frac{b_{rcrit}}{B_z}\right)_{DR-\theta} \propto n^{1/2} T^{1/8} B^{-5/4} R^{-1/2} \mathcal{P}^{3/8}, \quad (146)$$

$$\left(\frac{b_{rcrit}}{B_z}\right)_{DR-\phi} \propto n^0 T^{-1/8} B^{-3/4} R^{-1} \mathcal{P}^{5/8}. \quad (147)$$

It can be seen that the error-field penetration threshold exhibits no explicit dependence on the plasma density in regimes in which the change in the $\mathbf{E} \times \mathbf{B}$ frequency at the resonant surface is predominately due to a toroidal ion fluid velocity shift.^{7,13} On the other hand, the penetration threshold scales as $n^{1/2}$ in regimes in which the change in the $\mathbf{E} \times \mathbf{B}$ frequency at the resonant surface is predominately due to a poloidal ion fluid velocity shift. An $n^{1/2}$ scaling of the penetration threshold has been observed in a number of tokamak experiments.^{17,23,24,29,30} On the other hand, some experiments have reported

a linear scaling of the penetration threshold with n .^{16,19–22,31} The latter scaling is difficult to account for on the basis of the theory presented in this section.

F. Ohmic plasmas

In order to make further progress, we need to adopt a specific scaling model for the energy confinement timescale, τ_E . Consider Ohmic plasmas for which $\mathcal{P} = 1$. Let us adopt the following slightly modified version of the well-known neo-Alcator scaling law for low-density Ohmically heated tokamak plasmas:^{32–34}

$$\tau_E \propto n R^{13/4}. \quad (148)$$

(The modification is to ensure that the scaling law is consonant with the Connor–Taylor constraints, which require that $\Omega_i \tau_E \propto n^{\alpha_n} B^{2\beta} \times R^{2\alpha_n + (5/4)\alpha_B}$.²⁶) It follows that

$$\nu_* \propto \rho_*^{2/3} \beta_*, \quad (149)$$

$$T \propto B^{4/5} R^{1/2}. \quad (150)$$

Hence, we deduce that

$$\left(\frac{b_{\text{rcrit}}}{B_z}\right)_{\text{VR}-\varphi} \propto n^0 B^{-13/15} R^{-13/12} = n^{0.00} B^{-0.87} R^{-1.08}, \quad (151)$$

$$\left(\frac{b_{\text{rcrit}}}{B_z}\right)_{\text{VR}-\theta} \propto n^{1/2} B^{-7/6} R^{-11/24} = n^{0.50} B^{-1.17} R^{-0.46}, \quad (152)$$

$$\left(\frac{b_{\text{rcrit}}}{B_z}\right)_{\text{DR}-\theta} \propto n^{1/2} B^{-23/20} R^{-7/16} = n^{0.50} B^{-1.15} R^{-0.44}, \quad (153)$$

$$\left(\frac{b_{\text{rcrit}}}{B_z}\right)_{\text{DR}-\varphi} \propto n^0 B^{-17/20} R^{-17/16} = n^{0.00} B^{-0.85} R^{-1.06}. \quad (154)$$

Equations (151)–(154) confirm the n^0 scaling of the error-field penetration threshold in regimes in which the change in the $\mathbf{E} \times \mathbf{B}$ frequency at the resonant surface is predominately due to a toroidal ion fluid velocity shift, and the $n^{1/2}$ scaling when the change is predominately due to a poloidal ion fluid velocity shift. All regimes exhibit a roughly linear inverse scaling with increasing magnetic field-strength. On the other hand, the regimes in which the threshold scales as $n^{1/2}$ exhibit a weaker inverse scaling with machine size (roughly $R^{-1/2}$) than the regimes in which the threshold scales as n^0 (which exhibit an approximate R^{-1} scaling). (In fact, it is easily demonstrated, from the previous analysis, that if $b_{\text{rcrit}}/B_z \propto n^{\alpha_n} B^{2\beta} R^{\alpha_R}$ then $\alpha_R = 2\alpha_n + 1.25\alpha_B$.³¹)

G. L/H-mode plasmas

For externally heated (i.e., $\mathcal{P} > 1$) L/H-mode plasmas, we adopt the ITER89-P scaling law,³⁵ which yields

$$T \propto n^{-18/35} B^{41/35} R^{27/35} \mathcal{P}^{2/7}. \quad (155)$$

(Note that we are assuming that the plasma rotation remains diamagnetic, which means that we are not taking into account any significant injection of toroidal momentum into the plasma as a consequence of the external heating scheme. This assumption is reasonable for ITER.) Hence, we deduce that

$$\begin{aligned} \left(\frac{b_{\text{rcrit}}}{B_z}\right)_{\text{VR}-\varphi} &\propto n^{-3/35} B^{-169/210} R^{-218/210} \mathcal{P}^{53/84} \\ &= n^{-0.09} B^{-0.80} R^{-1.04} \mathcal{P}^{0.63}, \end{aligned} \quad (156)$$

$$\begin{aligned} \left(\frac{b_{\text{rcrit}}}{B_z}\right)_{\text{VR}-\theta} &\propto n^{2/7} B^{-85/84} R^{-29/84} \mathcal{P}^{19/42} \\ &= n^{0.29} B^{-1.01} R^{-0.35} \mathcal{P}^{0.45}, \end{aligned} \quad (157)$$

$$\begin{aligned} \left(\frac{b_{\text{rcrit}}}{B_z}\right)_{\text{DR}-\theta} &\propto n^{61/140} B^{-309/280} R^{-113/280} \mathcal{P}^{23/56} \\ &= n^{0.44} B^{-1.10} R^{-0.40} \mathcal{P}^{0.41}, \end{aligned} \quad (158)$$

$$\begin{aligned} \left(\frac{b_{\text{rcrit}}}{B_z}\right)_{\text{DR}-\varphi} &\propto n^{9/140} B^{-251/280} R^{-307/280} \mathcal{P}^{33/56} \\ &= n^{0.06} B^{-0.90} R^{-1.09} \mathcal{P}^{0.59}. \end{aligned} \quad (159)$$

It can be seen, by comparison with Eqs. (151)–(154), that the ITER89-P scaling law leads to a somewhat weaker scaling of the penetration threshold with density compared to that predicted by the neo-Alcator scaling law. On the other hand, the scalings of the penetration threshold with magnetic field-strength and machine size predicted by the two scaling laws are remarkably similar.

H. Comparison with experimental data

The most up to date and comprehensive study of the empirical scaling of the error-field penetration threshold with machine parameters in tokamak plasmas is that of Ref. 36. For $n=1$ modes in L/H-mode plasmas, it is found that

$$\frac{b_{\text{rcrit}}}{B_z} \propto n^{0.58 \pm 0.06} B^{-1.13 \pm 0.07} R^{+0.10 \pm 0.07}. \quad (160)$$

It can be seen, by comparison with Eqs. (156)–(159), that the experimental density and magnetic field-strength scalings of the penetration threshold are reasonably compatible with those predicted in the theoretical diffusive-resistive-poloidal (DR- θ) regime. On the other hand, the scaling with machine size is significantly different. All of the theoretical error-field penetration regimes lead to a negative scaling of the penetration threshold with machine size. On the other hand, the experimental scaling features a positive scaling with machine size. This discrepancy has significant consequences for ITER, which is much larger than present-day tokamaks. It should be noted that the experimental scaling with machine size is the most uncertain of all of the empirical scalings, because it is not possible to modify the major radius of a given tokamak. Hence, there are only as many effective data points as there are different machines in the data set. However, since most modern-day machines (e.g., DIII-D, EAST, JET) are of similar size, the size scaling of the penetration threshold is hypersensitive to data obtained from small tokamaks.

VIII. SUMMARY AND DISCUSSION

We have employed a slightly improved version of the four-field model derived in Ref. 8 to map out all of the two-fluid response regimes of a linear tearing layer that interacts with an externally generated resonant magnetic perturbation, in a large

aspect-ratio tokamak plasma, taking into account realistic levels of perpendicular particle transport. This analysis is a generalization of that presented in Ref. 7. Our results are summarized in Figs. 1 and 2. We find that there are four constant- ψ response regimes: the resistive-inertial, the viscous-resistive, the semi-collisional, and the diffusive-resistive. In addition, there are three nonconstant- ψ response regimes: the inertial, the viscous-inertial, and the diffusive-inertial. In general, both the number and the mutual arrangement of the various response regimes shown in Figs. 1 and 2 are different from those reported in Ref. 7. Moreover, these differences are a direct consequence of including realistic levels of perpendicular particle transport in the analysis. In particular, perpendicular particle diffusivity plays an explicit role in the diffusive-resistive and the diffusive-inertial response regimes.

We have employed our improved analysis to map out all of the growth-rate regimes of a non-interacting low-mode-number (but not $m = 1$) tearing mode in a large aspect-ratio tokamak plasma. We find that there are four growth-rate regimes: the resistive-inertial, the viscous-resistive, the semi-collisional, and the diffusive-resistive. The extents of these regimes are illustrated in Fig. 3. Realistic levels of perpendicular particle diffusion significantly modify the growth-rate in the resistive-inertial regime (which is the original Furth, Killeen, and Rosenbluth regime) and also affect the growth-rate in the diffusive-resistive regime. Perpendicular transport also suppresses spurious tearing mode branches in which the mode co-rotates with the MHD, or with the ion, fluid at the resonant surface, rather than with the electron fluid.

Finally, we have used our improved analysis to determine the scaling of the error-field penetration threshold with machine parameters in large aspect-ratio tokamak plasmas. We find that one particular error-field penetration regime (i.e., the diffusive-resistive-poloidal) leads to predicted scalings of the error-field penetration threshold with plasma density and magnetic field-strength that are reasonably consistent with the empirical scaling deduced from experimental data. On the other hand, all of the theoretical scalings predict a negative scaling of the penetration threshold with machine size, whereas the experimental data indicates a positive scaling. This discrepancy has significant consequences for ITER, which is much larger than present-day tokamaks.

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AUTHOR DECLARATIONS

Conflict of Interest

The author has no conflicts of interest.

DATA AVAILABILITY

The data that support the findings of this study are available from the corresponding author upon reasonable request.

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